APPENDIX 1

U. S. NUCLEAR REGULATORY COMMISSION

EMERGENCY ACTION LEVEL GUIDELINES
FOR NUCLEAR POWER PLANTS

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BASIS FOR EMERGENCY ACTION LEVELS FOR NUCLEAR POWER FACILITIES

Four classes of Emergency Action Levels are established which replace the classes in Regulatory Guide 1.101, each with associated examples of initiating conditions. The classes are:

Notification of Unusual Event

Alert

Site Area Emergency

General Emergency

The rationale for the notification and alert classes is to provide early and prompt notification of minor events which could lead to more serious consequences given operator error or equipment failure or which might be indicative of more serious conditions which are not yet fully realized. A gradation is provided to assure fuller response preparations for more serious indicators. The site area emergency class reflects conditions where some significant releases are likely or are occurring but where a core melt situation is not indicated based on current information. In this situation full mobilization of emergency personnel in the near site environs is indicated as well as dispatch of monitoring teams and associated communications. The general emergency class involves actual or imminent substantial core degradation or melting with the potential for loss of containment. The immediate action for this class is sheltering (staying inside) rather than evacuation until an assessment can be made that (1) an evacuation is indicated and (2) an evacuation, if indicated, can be completed prior to significant release and transport of radioactive material to the affected areas.

The example initiating conditions listed after the immediate actions for each class are to form the basis for establishment by each licensee of the specific plant instrumentation readings (as applicable) which, if exceeded, will initiate the emergency class.

Potential NRC actions during various emergency classes are given in NUREG-0728, Report to Congress: NRC Incident Response Plan. The NRC response to any notification from a licensee will be related to, but not limited by, the licensee estimate of severity; NRC will consider such other factors as the degree of uncertainty and the lead times required to position NRC response personnel should something more serious develop.

Prompt notification of offsite authorities is intended to indicate within about 15 minutes for the unusual event class and sooner (consistent with the need for other emergency actions) for other classes. The time is measured from the time at which operators recognize that events have occurred which make declaration of an emergency class appropriate.

State and/or Local Uffsite	Authority Actions	
	Licensee Actions	
	Class	

NOTIFICATION OF UNUSUAL EVENT Class Description

potential degradation of the level of safety of the plant. No Unusual events are in process or releases of radioactive material have occurred which indicate a monitoring are expected unless requiring offsite response or further degradation of safety systems occurs.

Purpose

step in any response later found to be necessary has been carried is to (1) assure that the first Purpose of offsite notification information and decisionmaking. staff to a state of readiness, out, (2) bring the operating and (3) provide systematic nandling of unusual events

Promptly inform State and/or local offsite authorities of nature of unusual condition as soon as discovered

Augment on-shift resources as needed 2

Assess and respond ლ Escalate to a more severe class, if appropriate 4.

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Close out with verbal summary to offsite authorities; followed by written summary within 24 hours ۍ ب

Provide fire or security assistance if requested

Escalate to a more severe class, if appropriate 2

Stand by until verbal closeout .

EXAMPLE INITIATING CONDITIONS: NOTIFICATION OF UNUSUAL EVENT

- I. Emergency Core Cooling System (ECCS) initiated and discharge to vessel
- 2. Radiological effluent technical specification limits exceeded
- 3. Fuel damage indication. Examples:
 - a. High offgas at BWR air ejector monitor (greater than 500,000 uci/sec; corresponding to 16 isotopes decayed to 30 minutes; or an increase of 100,000 uci/sec within a 30 minute time period)
 - High coolant activity sample (e.g., exceeding coolant technical specifications for iodine spike)
 - c. Failed fuel monitor (PWR) indicates increase greater than 0.1% equivalent fuel failures within 30 minutes
- 4. Abnormal coolant temperature and/or pressure or abnormal fuel temperatures outside of technical specification limits
- 5. Exceeding either primary/secondary leak rate technical specification or primary system leak rate technical specification
- 6. Failure of a safety or relief valve in a safety related system to close following reduction of applicable pressure
- 7. Loss of offsite power or loss of onsite AC power capability
- 8. Loss of containment integrity requiring shutdown by technical specifications
- 9. Loss of engineered safety feature or fire protection system function requiring shutdown by technical specifications (e.g., because of malfunction, personnel error or procedural inadequacy)
- 10. Fire within the plant lasting more than 10 minutes
- 11. Indications or alarms on process or effluent parameters not functional in control room to an extent requiring plant shutdown or other significant loss of assessment or communication capability (e.g., plant computer, Safety Parameter Display System, all meteorological instrumentation)
- 12. Security threat or attempted entry or attempted sabotage
- 13. Natural phenomenon being experienced or projected beyond usual levels
 - a. Any earthquake felt in-plant or detected on station seismic instrumentation
 - b. 50 year floor or low water, tsunami, hurricane surge, seiche
 - c. Any tornado on site
 - d. Any hurricane

- 14. Other hazards being experienced or projected
 - a. Aircraft crash on-site or unusual aircraft activity over facility
 - b. Train derailment on-site
 - c. Near or onsite explosion
 - d. Near or onsite toxic or flammable gas release
 - e. Turbine rotating component failure causing rapid plant shutdown
- 15. Other plant conditions exist that warrant increased awareness on the part of a plant operating staff or State and/or local offsite authorities or require plant shutdown under technical specification requirements or involve other than normal controlled shutdown (e.g., cooldown rate exceeding technical specification limits, pipe cracking found during operation)
- 16. Transportation of contaminated injured individual from site to offsite hospital
- 17. Rapid depressurization of PWR secondary side.

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Class

ALERT

Class Description

Events are in process or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant. Any releases expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

Purpose

Purpose of offsite alert is to (1) assure that emergency personnel are readily available to respond if situation becomes more serious or to perform confirmatory radiation monitoring if required, and (2) provide offsite authorities current status information.

Licensee Actions

- Promptly inform State and/or local authorities of alert status and reason for alert as soon as discovered
- 2. Augment resources and activate on-site Technical Support Center and on-site operational support center. Bring Emergency Operations Facility (EOF) and other key emergency personnel to standby status
- . Assess and respond
- Dispatch on-site monitoring teams and associated communications
- Provide periodic plant status updates to offsite authorities (at least every 15 minutes)
- Provide periodic meteorological assessments to offsite authorities and, if any releases are occurring, dose estimates for actual releases
- Escalate to a more severe class, if appropriate
- 8. Close out or recommend reduction in emergency class by verbal summary to offsite authorities followed by written summary within 8 hours of closeout or class reduction

State and/or Local Offsite Authority Actions

- Provide fire or security assistance if requested
- Augment resources and bring primary response centers and EBS to standby status
- Alert to standby status key emergency personnel including monitoring teams and associated communications
- 4. Provide confirmatory offsite radiation monitoring and ingestion pathway dose projections if actual releases substantially exceed technical specification limits
- Escalate to a more severe class, if appropriate
- Maintain alert status until verbal closeout or reduction of emergency class

EXAMPLE INITIATING CONDITIONS: ALERT

- 1. Severe loss of fuel cladding
 - a. High offgas at BWR air ejector monitor (greater than 5 ci/sec; corresponding to 16 isotopes decayed 30 minutes)
 - b. Very high coolant activity sample (e.g., 300 uci/cc equivalent of I-131)
 - c. Failed fuel monitor (PWR) indicates increase greater than 1% fuel failures within 30 minutes or 5% total fuel failures.
- 2. Rapid gross failure of one steam generator tube with loss of offsite power
- 3. Rapid failure of steam generator tubes (e.g., several hundred gpm primary to secondary leak rate)
- 4. Steam line break with significant (e.g., greater than 10 gpm) primary to secondary leak rate (PWR) or MSIV malfunction causing leakage (BWR)
- 5. Primary coolant leak rate greater than 50 gpm
- 6. Radiation levels or airborne contamination which indicate a severe degradation in the control of radioactive materials (e.g., increase of factor of 1000 in direct radiation readings within facility)
- 7. Loss of offsite power and loss of all onsite AC power (see Site Area Emergency for extended loss)
- 8. Loss of all onsite DC power (See Site Area Emergency for extended loss)
- 9. Coolant pump seizure leading to fuel failure
- 10. Complete loss of any function needed for plant cold shutdown
- 11. Failure of the reactor protection system to initiate and complete a scram which brings the reactor subcritical
- 12. Fuel damage accident with release of radioactivity to containment or fuel handling building
- 13. Fire potentially affecting safety systems
- 14. Most or all alarms (annunciators) lost
- 15. Radiological effluents greater than 10 times technical specification instantaneous limits (an instantaneous rate which, if continued over 2 hours, would result in about 1 mr at the site boundary under average meteorological conditions)
- 16. Ongoing security compromise

- 17. Severe natural phenomena being experienced or projected
 - a. Earthquake greater than OBE levels
 - b. Flood, low water, tsunami, hurricane surge, seiche near design levels
 - c. Any tornado striking facility
 - d. Hurricane winds near design basis level
- 18. Other hazards being experienced or projected
 - a. Aircraft crash on facility
 - b. Missile impacts from whatever source on facility
 - c. Known explosion damage to facility affecting plant operation
 - d. Entry into facility environs of uncontrolled toxic or flammable gases
 - e. Turbine failure causing casing penetration
- 19. Other plant conditions exist that warrant precautionary activation of technical support center and placing near-site Emergency Operations Facility and other key emergency personnel on standby
- 20. Evacuation of control room anticipated or required with control of shutdown systems established from local stations

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Class

SITE AREA EMERGENCY

Class Description

Events are in process or have occurred which involve actual or likely major failures of plant functions needed for protection of the public. Any releases not expected to exceed EPA Protective Action Guideline exposure levels except near site boundary.

Purpose

Purpose of the site area emergency declaration is to (1) assure that response centers are manned, (2) assure that monitoring teams are dispatched, (3) assure that personnel required for evacuation of near-site areas are at duty stations if situation becomes more serious, (4) provide consultation with offsite authorities, and (5) provide updates for the public

Licensee Actions

- Promptly inform State and/or local offsite authorities of site area emergency status and reason for emergency as soon as discovered
- Augment resources by activating on-site Technical Support Center, on-site operational support center and near-site Emergency Operations Facility (EOF)
- 3. Assess and respond
- 4. Dispatch on-site and offsite monitoring teams and associated communications
- 5. Dedicate an individual for plant status updates to offsite authorities and periodic pressure briefings (perhaps joint with offsite authorities)
- Make senior technical and management staff onsite available for consultation with NRC and State on a periodic basis
- 7. Provide meteorological and dose estimates to offsite authorities for actual releases via a dedicated individual or automated data transmission
- 8. Provide release and dose projections based on available plant condition information and foreseeable contingencies
- Escalate to general emergency class, if appropriate

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10. Close out or recommend reduction in emergency class by briefing of offsite authorities at EOF and by phone followed by written summary within 8 hours of closeout or class reduction

State and/or Local Offsite Authority Actions

- 1. Provide any assistance requested
- If sheltering near the site
 is desirable, activate public
 notification system within
 at least two miles of the plant
- Provide public within at least about 10 miles periodic updates on emergency status
- Augment resources by activating primary response centers
- Dispatch key emergency personnel including monitoring teams and associated communications
- Alert to standby status other emergency personnel (e.g., those needed for evacuation) and dispatch personnel to near-site duty stations
- 7. Provide offsite monitoring results to licensee, DOE and others and jointly assess them
- 8. Continuously assess information from licensee and offsite monitoring with regard to changes to protective actions already initiated for public and mobilizing evacuation resources
- Recommend placing milk animals within 2 miles on stored feed and assess need to extend distance
- Provide press briefings, perhaps with licensee
- Escalate to general emergency class, if appropriate
- 12. Maintain site area emergency status until closeout or reduction of emergency class

EXAMPLE INITIATING CONDITIONS: SITE AREA EMERGENCY

- 1. Known loss of coolant accident greater than makeup pump capacity
- Degraded core with possible loss of coolable geometry (indicators should include instrumentation to detect inadequate core cooling, coolant activity and/or containment radioactivity levels)
- 3. Rapid failure of steam generator tubes (several hundred gpm leakage) with loss of offsite power
- 4. BWR steam line break outside containment without isolation
- 5. PWR steam line break with greater than 50 gpm primary to secondary leakage and indication of fuel damage
- 6. Loss of offsite power and loss of onsite AC power for more than 15 minutes
- 7. Loss of all vital onsite DC power for more than 15 minutes
- 8. Complete loss of any function needed for plant hot shutdown
- 9. Transient requiring operation of shutdown systems with failure to scram (continued power generation but no core damage immediately evident)
- 10. Major damage to spent fuel in containment or fuel handling building (e.g., large object damages fuel or water loss below fuel level)
- 11. Fire compromising the functions of safety systems
- 12. Most or all alarms (annunciators) lost and plant transient initiated or in progress
- 13. a. Effluent monitors detect levels corresponding to greater than 50 mr/hr for 1/2 hour or greater than 500 mr/hr W.B. for two minutes (or five times these levels to the thyroid) at the site boundary for adverse meteorology
 - b. These dose rates are projected based on other plant parameters (e.g., radiation level in containment with leak rate appropriate for existing containment pressure) or are measured in the environs
 - c. EPA Protective Action Guidelines are projected to be exceeded outside the site boundary
- 14. Imminent loss of physical control of the plant
- 15. Severe natural phenomena being experienced or projected with plant not in cold shutdown
 - a. Earthquake greater than SSE levels

- b. Flood, low water, tsunami, hurricane surge, seiche greater than design levels or failure of protection of vital equipment at lower levels
- c. Sustained winds or tornadoes in excess of design levels
- 16. Other hazards being experienced or projected with plant not in cold shutdown
 - a. Aircraft crash affecting vital structures by impact or fire
 - b. Severe damage to safe shutdown equipment from missiles or explosion
 - c. Entry of uncontrolled flammable gases into vital areas. Entry of uncontrolled toxic gases into vital areas where lack of access to the area constitutes a safety problem
- 17. Other plant conditions exist that warrant activation of emergency centers and monitoring teams or a precautionary notification to the public near the site
- 18. Evacuation of control room and control of shutdown systems not established from local stations in 15 minutes

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GENERAL EMERGENCY

Class Description

potential for loss of containment integrity. Releases can Events are in process or have occurred which involve actual or imminent substantial core exceed EPA Protective Action degradation or melting with Guideline exposure levels be reasonably expected to offsite for more than the immediate site area.

Purpose

Purpose of the general emergency declaration is to (1) initiate predetermined protective actions measures as indicated by actual ments, (3) initiate additional information from licensee and offsite organization measurefor the public, (2) provide (5) provide updates for the provide consultation with continuous assessment of offsite authorities and or potential releases, public through offsite authorities

Licensee Actions

- Promptly inform State and local offsite aut orities of general emergency status discovered (Parallel notification of and reason for emergency as soon as State/local)
- site Emergency Operations Facility (EOF) Augment resources by activating on-site operational support center and near-Technical Support Center, on-site ς,
- Assess and respond

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- Dispatch on-site and offsite monitoring teams and associated communications 4.
- periodic press briefings (perhaps joint with offsite authorities) Dedicate an individual for plant status updates to offsite authorities and 5
- Make senior technical and management staff onsite available for consultation with NRC and State on a periodic basis တ်
- Provide meteorological and dose estimates releases via a dedicated individual or automated data transmission to offsite authorities for actual 7
- information and foreseeable contingencies Provide release and dose projections based on available plant condition φ.
- authorities at EOF and by phone followed by written summary within 8 hours of emergency class by briefing of offsite Close out or recommend reduction of closeout or class reduction 9.

State and/or Local Offsite Authority Actions

- Provide any assistance requested
- Activate immediate public status and provide public notification of emergency periodic updates 2
- wind and assess need to extend mile radius and 5 miles downdistances. Consider advisa-bility of evacuation (projected time available vs. estimated evacuation times) Recommend sheltering for 2 ω,
- Augment resources by activating primary response centers 4
- Dispatch key emergency personnel including monitoring teams and associated communications S.
- within 5 mile radius and alert all others to standby status results to licensee, DOE and personnel to duty stations Provide offsite monitoring Dispatch other emergency ė 7
- tion from licensee and offsite changes to protective actions already initiated for public Continuously assess informamonitoring with regard to and mobilizing evacuation resources ထိ

others and jointly assess them

- Recommend placing milk animals within 10 miles on stored feed and assess need to extend distance 6
- Provide press briefings, perhaps with licensee 10°
- status until closeout or reduction of emergency class Maintain general emergency =

EXAMPLE INITIATING CONDITIONS: GENERAL EMERGENCY

- 1. a. Effluent monitors detect levels corresponding to 1 rem/hr W.B. or 5 rem/hr thyroid at the site boundary under actual meteorological conditions
 - b. These dose rates are projected based on other plant parameters (e.g., radiation levels in containment with leak rate appropriate for existing containment pressure with some confirmation from effluent monitors) or are measured in the environs

Note: Consider evacuation only within about 2 miles of the site boundary unless these site boundary levels are exceeded by a factor of 10 or projected to continue for 10 hours or EPA Protective Action Guideline exposure levels are predicted to be exceeded at longer distances

- 2. Loss of 2 of 3 fission product barriers with a potential loss of 3rd barrier, (e.g., loss of primary coolant boundary, clad failure, and high potential for loss of containment)
- 3. Loss of physical control of the facility

Note: Consider 2 mile precautionary evacuation

- 4. Other plant conditions exist, from whatever source, that make release of large amounts of radioactivity in a short time period possible, e.g., any core melt situation. See the specific PWR and BWR sequences below.
 - Notes: a. For core melt sequences where significant releases from containment are not yet taking place and large amounts of fission products are not yet in the containment atmosphere, consider 2 mile precautionary evacuation. Consider 5 mile downwind evacuation (45° to 90° sector) if large amounts of fission products (greater than gap activity) are in the containment atmosphere. Recommend sheltering in other parts of the plume exposure Emergency Planning Zone under this circumstance.
 - b. For core melt sequences where significant releases from containment are not yet taking place and containment failure leading to a direct atmospheric release is likely in the sequence but not imminent and large amounts of fission products in addition to noble gases are in the containment atmosphere, consider precautionary evacuation to 5 miles and 10 mile downwind evacuation (450 to 90° sector).
 - c. For core melt sequences where large amounts of fission products other than noble gases are in the containment atmosphere and containment failure is judged imminent, recommend shelter for those areas where evacuation cannot be completed before transport of activity to that location.

d. As release information becomes available adjust these actions in accordance with dose projections, time available to evacuate and estimated evacuation times given current conditions.

5. Example PWR Sequences

- a. Small and large LOCA's with failure of ECCS to perform leading to severe core degradation or melt in from minutes to hours. Ultimate failure of containment likely for melt sequences. (Several hours likely to be available to complete protective actions unless containment is not isolated)
- b. Transient initiated by loss of feedwater and condensate systems (principal heat removal system) followed by failure of emergency feedwater system for extended period. Core melting possible in several hours. Ultimate failure of containment likely if core melts.
- c. Transient requiring operation of shutdown systems with failure to scram which results in core damage or additional failure of core cooling and makeup systems (which could lead to core melt)
- d. Failure of offsite and onsite power along with total loss of emergency feedwater makeup capability for several hours. Would lead to eventual core melt and likely failure of containment.
- Small LOCA and initially successful ECCS. Subsequent failure of containment heat removal systems over several hours could lead to core melt and likely failure of containment.
- NOTE: Most likely containment failure mode is melt-through with release of gases only for dry containment; quicker and larger releases likely for ice condenser containment for melt sequences. Quicker releases expected for failure of containment isolation system for any PWR.

6. Example BWR Sequences

- a. Transient (e.g., loss of offsite power) plus failure of requisite core shut down systems (e.g., scram). Could lead to core melt in several hours with containment failure likely. More severe consequences if pumps trip does not function.
- b. Small or large LOCA's with failure of ECCS to perform leading to core melt degradation or melt in minutes to hours. Loss of containment integrity may be imminent.
- c. Small or large LOCA occurs and containment performance is unsuccessful affecting longer term success of the ECCS. Could lead to core degradation or melt in several hours without containment boundary.

- d. Shutdown occurs but requisite decay heat removal systems (e.g., RHR) or non-safety systems heat removal means are rendered unavailable. Core degradation or melt could occur in about ten hours with subsequent containment failure.
- 7. Any major internal or external events (e.g., fires, earthquakes, substantially beyond design basis) which could cause massive common damage to plant systems resulting in any of the above.

METEOROLOGICAL CRITERIA FOR EMERGENCY PREPAREDNESS AT OPERATING NUCLEAR POWER PLANTS

Introduction

10 CFR Part 50.47 requires that the Emergency Plan shall provide "(A)dequate methods, systems, and equipment for assessing and monitoring actual or potential offsite consequences of a radiological emergency condition ..."

The basic functions needed to comply with the meteorological aspects of these requirements are:

- 1. A capability for making meteorological measurements.
- 2. A capability for making near real-time predictions of the atmospheric effluent transport and diffusion.
- 3. A capability for remote interrogation of the atmospheric measurements and predictions by appropriate organizations.

A staged schedule is provided in Annex 1 to this appendix for implementation of the meteorological elements addressing emergency preparedness requirements.

Meteorological Measurements

The emergency facilities and equipment as stated in Appendix E to 10 CFR Part 50 shall include "(E)quipment for determining the magnitude of and for continuously assessing the impact of the release of radioactive materials to the environment." To address this requirement, in part, the nuclear power plant operator shall have meteorological measurements from primary and backup systems.

Each site with an operating nuclear power plant shall have a primary meteorological measurements system. The primary system shall produce current and record historical local meteorological data. These data will provide a means to estimate the dispersion of radioactive material due to accidental radioactive releases to the atmosphere by the plant. The acceptance criteria for meteorological measurements are described in the proposed Revision 1 to U. S. NRC Regulatory Guide 1.23.

Each site with an operating nuclear power plant shall have a viable backup meteorological measurements system. The backup system shall provide meteorological information when the primary system is out of service and, thus, assurance that basic meteorological information is available during and immediately following an accidental airborne radioactivity release. The acceptance criteria for the backup meteorological measurements system are described in the proposed Revision 1 to U. S. NRC Regulatory Guide 1.23.

Atmospheric Transport and Diffusion Assessment

Appendix E to 10 CFR Part 50 states that "(T)he means to be used for determining the magnitude of and for continually assessing the impact of the release of radioactive materials shall be described ..." To address this requirement, in part, all licensees with operating nuclear power plants shall provide the description of their system for making current, site-specific estimates and predictions of atmospheric effluent transport and diffusion during and immediately following an accidental airborne radioactivity release from the nuclear power plant. The purpose of these predictions is to provide an input to the assessment of the consequences of accidental radioactive releases to the atmosphere and to aid in the implementation of emergency response decisions.

Near real-time, site-specific atmospheric transport and diffusion models shall be used when accidental airborne radioactive releases occur. Two classes of models are appropriate. The first, Class A, is a model and calculational capability which can produce initial transport and diffusion estimates for the plume exposure EPZ within 15 minutes following the classification of an incident. The second, Class B, is a numerical model which represents the actual spatial and temporal variations of plume distribution and can provide estimates of deposition and relative concentration of radioactivity within the plume exposure and ingestion EPZs for the duration of the release.

The Class A model shall use actual 15 minute average meteorological data from the meteorological measurements systems maintained by the licensee. The selected data shall be indicative of the conditions within the plume exposure EPZ. The Class A model shall provide calculations or relative concentrations (X/Q) and transit times within the plume exposure EPZ. Atmospheric diffusion rates shall be based on atmospheric stability as a function of site-specific terrain conditions. Site-specific local climatological effects on the trajectories, such as seasonal, diurnal, and terrain-induced flows shall be included. Source characteristics (release mode, and building complex influence) shall be factored into the model. The output from the Class A model shall include the plume dimensions and position, and the location, magnitude, and arrival time of (1) the peak relative concentration and (2) the relative concentrations at appropriate locations. The bases and justification for these model(s) and input data shall be documented. The performance and limitations of the model(s) shall also be included in the documentation.

The essential elements of the input, of model components, and of output to be incorporated in the Class A model are given to provide guidance for meteorological system implementation. Additional guidance will be prepared to outline the staff position on dose assessment capabilities to be used for emergency response.

Remote Interrogation

Appendix E to 10 CFR Part 50 states that there shall be "(P)rovisions for communications among the nuclear power reactor control room, the onsite technical support center and the near-site emergency operations facility "There shall also be "(P)rovisions for communications by the licensee with the NRC Headquarters and the appropriate NRC Regional Office Operations Center from the nuclear power reactor control room, the onsite technical support center, and the near-site emergency operations facility" and "... among the nuclear facility, the principal State and local emergency operations centers"

To address this requirement with respect to the meteorological information, all systems producing meteorological data and effluent transport and diffusion estimates at sites with operating nuclear power plants shall have the capability of being remotely interrogated. This will provide current meteorological data and transport and diffusion estimates to the licensee, emergency response organizations, and the NRC staff, on-demand, during emergency situations.

Proposed Revision 1 to Regulatory Guide 1.23 identifies the meteorological data that shall be available. The information that shall be available from the transport and diffusion assessment include the model outputs, input variables, model identification and data source information, plant identification, and data from other sources, as available.

The capability to make transport and diffusion calculations with specific inputs shall be provided. The primary and backup communications systems shall have a data transmission rate of 1200 BAUD and the rate(s) and other specifications indicated in proposed Revision 1 to Regulatory Guide 1.23.

Documentation for procedures to access and use the system shall be provided to the emergency response organizations and the NRC, and shall be available in the control room, the Technical Support Center (TSC) and the Emergency Operations Facility (EOF).

ANNEX 1 TO APPENDIX 2

ADDRESSING EMERGENCY PLANNING RULES

Schedule for Operating Reactors -- For operating reactors the following implementation milestones shall be met to address the functional requirements.

Milestones are numbered and tagged with the following code; <u>a-date</u>, <u>b-activity</u>, <u>c-minimum</u> acceptance criteria. They are as follows:

- (1) a. January 2, 1981
 - b. Submittal of radiological emergency response plans
 - c. A description of the emergency plan which addresses the meteorological functions shall be provided
- (2) a. March 1, 1981
 - b. Submittal of implementing procedures
 - c. Methods, systems, and equipment to assess and monitor actual or potential offsite consequences of a radiological emergency condition shall be provided
- (3) a. April 1, 1981
 - b. Implementation of radiological emergency response plans
 - c. Three functions of Appendix 2 with the exception of the Class B model of the assessment capability

Alternative to milestone (3) requiring compensating actions:

A meteorological measurements system which is consistent with the existing technical specifications as the baseline or a primary system and/or a backup system of Appendix 2, or two independent backup systems shall provide the basic meteorological parameters (wind direction and speed and an indicator of atmospheric stability) on display in the control room. An operable dose calculational methodology (DCM) shall be in use in the control room and at appropriate emergency response facilities. The following compensating actions shall be taken by the licensee for this alternative:

- (i) if only a primary or a backup system is in use:
 - The licensee (a person who will be responsible for making offsite dose projections) shall check communications with the cognizant National Weather Service (NWS) first order station and NWS forecasting station on a monthly basis to ensure that routine meteorological observations and forecasts can be accessed.
 - The licensee shall calibrate the meteorological measurements at a frequency no less than quarterly and identify a readily available source of meteorological data (characteristic of site conditions) to which they can gain access during calibration periods.
 - During conditions of measurements system unavailability, an alternate source of meteorological data which is characteristic of site conditions shall be identified to which the licensee can gain access.

- O The licensee shall maintain a site inspection schedule for evaluation of the meteorological measurements system at a frequency no less than weekly.
- O It shall be a reportable occurrence if the meteorological data unavailability exceeds the goals outlined in Proposed Revision 1 to Regulatory Guide 1.23 on a quarterly basis.
- (ii) The portion of the DCM relating to the transport and diffusion of gaseous effluents shall be consistent with the characteristics of the Class A model outlined in the assessment capability of Appendix 2.
- (iii) Direct telephone access to the individual responsible for making offsite dose projections (Appendix E to 10 CFR Part 50(IV)(A)(4)) shall be available to the NRC in the event of a radiological emergency. Procedures for establishing contact and identification of contact individuals shall be provided as part of the implementing procedures.

This alternative shall not be exercised after July 1, 1982. Further, by July 1, 1981, a functional description of the upgraded capabilities and schedule for installation and operation shall be provided (see milestones 4 and 5).

- (4) a. April 1, 1982
 - b. Installation of Emergency Response Facility meteorological hardware and software

- c. Three functions of Appendix 2, with exception of the Class B model of the assessment capability
- (5) a. July 1, 1982
 - b. Full operation of milestone 4
 - c. The Class A model (designed to be used out to the plume exposure EPZ) may be used in lieu of a Class B model out to the ingestion EPZ. Compensating actions to be taken for extending the application of the Class A model out to the ingestion EPZ include access to supplemental information (meso and synoptic scale) to apply judgment regarding intermediate and long-range transport estimates. The distribution of meteorological information by the licensee should be as follows by July 1, 1982:

Meteorological Information	CR	TSC	EOF	NRC and Emergency Response Organizations
Basic Met. Data (e.g., 1.97 Parameters)	Х	Х	Χ	X (NRC)
Full Met. Data (1.23 Parameters)		Х	Χ	Х
DCM (for Dose Projections)	Х	χ	Χ	X
Class A Model (to Plume Exposure EPZ)	Χ	Χ	χ	Х
Class B Model or Class A Model (to Ingestion EPZ)		Х	X	Х

- (6) a. July 1, 1982 or at the time of the completion of milestone 5, whichever is sooner
 - b. Mandatory review of the DCM by the licensee

- c. Any DCM in use should be reviewed to ensure consistency with the operational Class A model. Thus, actions recommended during the initial phases of a radiological emergency would be consistent with those after the TSC and EOF are activated
- (7) a. September 1, 1982
 - b. Description of the Class B model provided to the NRC
 - c. Documentation of the technical bases and justification for selection of the type Class B model by the licensee with a discussion of the site-specific attributes
- (8) a. June 1, 1983
 - b. Full operation of the Class B model
 - c. Class B model of the assessment capability of Appendix 2

O Schedule for Near-Term OLs

For applicants for an operating license at least milestones 1, 2, and 3 shall be met prior to the issuance of an operating license. Subsequent milestones shall be met by the same dates indicated for operating reactors. For the alternative to milestone 3, the meteorological measurements system shall be consistent with the NUREG-75/087, "Standard Review Plan For the Review of Safety Analysis Reports for Nuclear Power Plants," Section 2.3.3 program as the baseline or primary system and/or backup system.